Research Article

Research of Potential Production ^{94m}Tc in Medical Cyclotron

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Abstract

To expand the spectrum of used radiopharmaceuticals, it is proposed to obtain a positronemitting isotope of technetium 94m Tc. The intention of this work is to research the possibility of producing various technetium isotopes on a medical cyclotron. For this purpose, we carried out a series of irradiations of an aqueous solution of molybdenum of natural isotopic composition with protons of 11 MeV energy. After technetium isolation, results were analyzed on a γ -spectrometer. 511 keV gamma-ray line was obtained.

Introduction

Positron emission tomography combined with computed tomography (PET/CT) is a highly informative diagnostic method in various areas of medicine, such as oncology, and cardiology, as well as in the diagnosis of inflammatory processes. In this regard, it is important to introduce new positron radionuclides for the synthesis of radiopharmaceuticals (RP), which will have a number of advantages over other RP labeled with widely used radionuclides ¹⁸F, ¹¹C, and ¹³N. One of the promising positrons (β +) emitters is the technetium isotope ^{94m}Tc, which has optimal nuclear-physical characteristics for use in PET, such as a positron branching ratio of 72% and a halflife of 52 min, allowing the synthesis of RP. The broad potential of ^{94m}Tc is due to the possibility of using existing commercial kits designed for the manufacture of RP from generator 99mTcpertechnetate [1]. For example, ^{94m}Tc-sestamibi can be used as a myocardial perfusion agent [2]. Scientists from the USA conducted comparative research on eight patients with a history of myocardial infarction with radiopharmaceuticals ¹³N-ammonia and ^{94m}Tc-sestamibi [3]. The authors did not reveal significant differences in the accumulation of RPs in the heart muscle, but the administered dose of 94mTc-sestamibi is lower than that of ¹³N-ammonia. In the same way, the technetium isotope 94mTc can be used for a wide range of radiopharmaceuticals as a radioactive label instead of 99mTc to obtain PET images. There are many methods to obtain ^{94m}Tc. However, the most suitable method for practical work is to irradiate a stable isotope of molybdenum ⁹⁴Mo with protons. The reaction ${}^{94}Mo(p,n){}^{94m}Tc$ gives out a high yield of the target radionuclide and a low content of isotopic impurity 94gTc about 6% - 10% [4]. In addition, ⁹⁴ Tc has a short half-life of 293 min

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and is itself a partial positron emitter (positron branching ratio 11%), so it will not create excess radiation exposure to patients. However, this method works with proton energy from 8 to 13 MeV only. Scientists from Germany conducted a study and found that above 13 MeV, the probability of a nuclear reaction with releases two neutrons from ⁹⁴Mo and the formation of isotopes ^{93g,m}Tc begins to increase rapidly, which is shown in the graph in Figure 1 [5]. Accordingly, the production of ^{94m}Tc by the nuclear reaction ⁹⁴Mo(p,n) ^{94m}Tc is possible only at low-energy accelerators, such as the medical cyclotron Siemens RDS-111 with a proton energy of 11 MeV. It also takes into consideration what is this cyclotron, as well as most other medical cyclotrons, designed for irradiation of liquid and gas target substances only. That's why molybdenum



must be used in the form of an aqueous solution. In addition, there are methods for relatively rapid extraction of technetium from the liquid phase [6,7].

Materials and methods

The essence of the experiment was to produce positronemitting ^{94m}Tc using the capabilities of a small radiochemical laboratory at the PET center. To achieve this, we faced 4 tasks:

- a) Selection of the composition of a molybdenum solution for irradiation on a cyclotron;
- b) Selection and programming of the target device;
- c) Development of a method for purifying Tc from positron-emitting isotopes in an irradiated solution;
- d) Definition of qualitative analysis of the obtained product for the technetium isotope composition.

- Preparation of liquid target solution

In this research, it is irradiated molybdenum of natural isotopic composition (Table 1).

As a target substance was used, an aqueous solution of potassium molybdate with a molybdenum concentration of about 100 mg/ml, was obtained by dissolving dry molybdenum (VI) oxide in 1M KOH.

- Cyclotron targets

Work on the production of technetium was carried out on the Siemens RDS-111 cyclotron. The target device is a modified standard TCU designed to produce radionuclide nitrogen-13. This allows the entire cycle of obtaining radionuclides to be carried out automatically. The operating principle of this device is shown schematically in Figure 2. The HPLC pump with a capacity of 1.5 ml/min feeds a solution of the target substance into the volume 3 ml aluminum target



through a mechanical valve set to a system pressure of 300 PSI and the pump constantly supports it during irradiation. After irradiation, a remotely controlled valve opens and the pump flows over the target solution into a hot box in the laboratory.

- Purification of technetium isotopes from radionuclide impurities

The produced technetium is stabilized in the chemical form of an aqueous solution of pertechnetate. The main part of radionuclide impurities to technetium are positron-emitting radionuclides, which are obtained from water by reactions ¹⁸O(p;n)¹⁸F ($T_{1/2}$ =110 min) and ¹⁶O(p; α)¹³N ($T_{1/2}$ =10 min). For this research, the obtained solution is delayed until ¹³N is fully decayed for at least 2 hours. ¹⁸F is formed as fluoride anions and is removed from the solution by precipitation with CaF₂ as an isotopic carrier:

 $Ca^{2+} + {}^{19}F^{-} + {}^{18}F^{-} = Ca^{19}F^{18}F\downarrow; (Ca^{2+} + TcO_{a}^{-} \neq).$

- Radioisotope measurements by γ-spectroscopy

A 100 μ l sample with an activity of about 1 MBq was added to a 3 ml vial of water and mixed thoroughly. The radionuclide composition was assessed using a γ -spectrometer HPGe with detector GEM30, manufacturer ORTEC. Since positrons annihilate with the emission of γ -quantum with energy 511 keV, then along this line it is possible to identify positron emitters including ^{94m}Tc, which does not have its own unique γ -line. As can be seen from Table 2, other isotopes of technetium have unique γ -energy lines. Shows itself especially well ^{99m}Tc, having a single, separate from other isotopes γ -line 140,45 keV.

Results and discussion

Isotopes technetium production

In the work, several irradiations with a current of 30 μ A were performed from 5 to 30 minutes. The pressure in the target during irradiation was constantly maintained at 300 PSI. Nuclear reactions with molybdenum isotopes can generate the radionuclides, presented in Table 3.

Table 2: γ -energy lines (keV) used for identification of technetium isotopes.												
^{99m} Tc	⁹⁵ Tc	β+	^{94g} Tc	⁹⁵ Tc	^{96m} Tc	^{96g} Tc	^{94g} Tc	^{94m,g} Tc	⁹⁵ Tc	⁹⁵ Tc		
140,45	204,14	511,4	703,3	766,6	779,06	813,4	850,7	872,1	948,8	1075		
Table 3: Nuclear reactions for the production of the different Tc isotopes and their specification [8,9].												
Reaction				Halt	f-life	Decay						
⁹⁴ Mo(p,n) ^{94g} Tc				293	min	87,9% EC; 11,1% β ⁺						
⁹⁴ Mo(p,n) ^{94m} Tc				52	min	29,8% EC; 72,2% β ⁺						
⁹⁵ Mo(p,n) ⁵ Tc				1200) min	100% EC						
⁹⁶ Mo(p,n) ^{96g} Tc				6163 min		100% EC						
⁹⁶ Mo(p,n) ^{96m} Tc				52	min	98% IT; 2% IC						
¹⁰⁰ Mo(p,2n) ^{99m} Tc				360 min		88% IT; 12% IC						



Accordingly, if we use isotopically pure Mo, we will obtain the only isotope we need, for example, ^{94m}Tc from ⁹⁴Mo. This will also ensure high yields of the target radionuclide from irradiation. The only difficulty is the high cost of isotopically pure molybdenum. And since the physical consumption of molybdenum for the production of technetium is very low, it is possible to regenerate molybdenum for its repeated irradiation.

Pertechnetate isolation

2 ml of 0.1 M NaF was pre-added to the vial prepared for collecting the target substance. After unloading the irradiated solution and mixing, 0.5 M CaCl₂ was added dropwise to the solution until a white precipitate completely formed. It was then passed through a sterilizing syringe nozzle with 0.22 μ m pores, ¹⁸F together with precipitate was retained on the filter, and TcO₄ passed into the filtrate. More effective ¹⁸F-fluoride removal is also possible, if the CaCl2 solution is placed in advance, and the isotopic carrier NaF is added after unloading the target substance.

Results of γ-spectroscopy

After cleaning from ¹⁸F and a delay of more than 2 hours for decay of ¹³N, the samples were analyzed on an γ -spectrometer. Spectrum was obtained and deciphered in the form of a diagram in Figure 3. There is a clean line at 140 keV, belonging to the isotope ^{99m}Tc, and line 511 κ B, belonging to positron emitters. As well as lines belonging to other isotopes of technetium. It turns out that this method can be used for the production of ^{99m}Tc from stable ¹⁰⁰Mo, which has a great advantage, compared to technetium from the generator, in the absence of the long-lived parent radionuclide ⁹⁹Mo [10].

Dynamic γ-spectroscopy by the line 511 keV

One of the samples was analyzed on an γ -spectrometer every 30 minutes. Based on the obtained data was done graphic representation of the decay rate of positron emitters in the sample (Figure 4). A comparative analysis was carried out with the decay rate of ^{94m}Tc, taking the coinciding points at 30 and 60 minutes as the calculated activity. As can be seen from the graphic, the decay of the sample almost coincides with the decay of ^{94m}Tc, this indirectly confirms that among positron emitters ^{94m}Tc gives the largest share to the line 511 keV. Additionally, a graphic was done of the decay rate of ¹⁸F, taking as the calculated value activity the difference between «decay of positron-emitting r/n» and «decay of 94mTc» in the last point of the graph. The decay rate of ¹³N is taken as the calculated value activity of the difference between decay graphics in the first point of the graph. As can be seen, these graphics are far from the main one, and therefore their content is at the of small impurities.

For further scientific work in this direction, it is necessary first of all refine the separation methodology pertechnetate anions not only from fluoride but also from ¹³N in anionic form. After which, to synthesize the radiopharmaceuticals, it will be necessary to construct a new or redesign already existing





Энергия, кэВ	1	2	3	4	5	6	7	8	9	10	11
	22-05-24 14:45	22-05-24 15:04	22-05-24 15:25	22-05-24 15	2 22 05 24 16-19	22 05 24 16-27	22 05 24 17-09	22.05.24.17.25	22 05 24 19-02	22 05-24 18:20	23-05-24 11:32
	0.00	0.32	0.67	0.97	140 🤹			itron omitting	radionu elidos in	3.58	20.78
ВРЕМЯ (МИН)	0.00	19.00	40.00	58.00	Ν		-uecay or pos	sitron emitting	autonuclides in	215.00	
140.45	9.1	9.06	8.62	8.33	120		sample			6.15	0.83
204	0.43	0.22	0.22	0.23			- do cour of 0.4			0.22	0.21
391	215.00	196.00	175.00	157.00							
481	0.65	0.46	0.34	0.26							
511.4	139.3	88.4	62.1	49.2		decay of 18F				10.02	
703.3	3.73	3.66	3.49	3.33						2.22	0.21
719	0.38	0.32	0.25	0.16							
766.6	10.78	10.39	10.38	10.3	5 60		do any of 12			9.27	5.02
779.06	4.47	4.36	4.14	3.95			decay of 13	N		3.38	2.8
813.4	1.98	2.18	2.31	2.35	40					2.51	
850.7	5.56	5.57	5.69	5.35			8			4.94	2.8
872.1	35	27.8	21.8	17.7	20		0			3.71	0.148
876	0.47	0.29	0.23	0.2		A		o o			
917	0.22	0.21	0.22	0.21	0		_			0.133	
948.8	0.18	0.18	0.16	0.19	0	50	100	150	200	0.135	0.094
993	0.65	0.52	0.38	0.33			Minute	s			
1075	0.31	0.31	0.27	0.31	0.3	0.33	0.29	0.26	0.24	0.28	0.138
1128	0.31	0.32	0.29	0.35	0.33	0.33	0.35	0.31	0.31	0.39	0.31
1197	0.23	0.17	0.094	0.08	0.069						
1202	0.78	0.66	0.41	0.34	0.21	0.2	0.137	0.088	0.059		
1365	0.34	0.35	0.28	0.26	0.24		0.191	0.193	0.179	0.171	
1384	0.076	0.063									
1524	1.01	0.85	0.68	0.52	0.29	0.24	0.182	0.071	0.073		
	105.84	82.16	62.10	48.85	30.64	23.78	15.73	12.54	7.56	6.03	
	15.50	13.75	12.05	10.76	8.63	7.65	6.30	5.66	4.45	4.00	
	17.96	4.81	1.12	0.32	0.03	0.01	0.00	0.00	0.00	0.00	

Figure 4: Comparison of decays +¦+-emitting radionuclides at a frequency of 511 keV.

automated radiochemistry synthesizer. The synthesis itself is hypothetically ordinary – useful RPs, labeled with the isotope ^{99m}Tc, and methods of their analysis are used everywhere. To obtain ^{94m}Tc without other isotopes of technetium expensive isotopically pure ⁹⁴Mo is required. And the main problem is allocation with minimal losses of molybdenum from irradiated solution. Without this, the research direction will not be economically justified.

Conclusion

This paper demonstrates the possibility of obtaining various technetium isotopes in a liquid target device of a lowenergy proton accelerator. Particular attention should be paid to the positron-emitting isotope ^{94m}Tc. The optimal half-life of 52 minutes allows chemistry reactions with ^{94m}Tc to obtain radiopharmaceuticals in PET diagnostics. In the future, if the irradiation parameters are optimized and isotopically pure ⁹⁴Mo is used in the target substance, it will be possible to obtain high yields of the target radionuclide.

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